



AREVA



The Fukushima Daiichi Incident

1. Plant Design
2. Accident Progression
3. Radiological releases
4. Spent fuel pools
5. Sources of Information

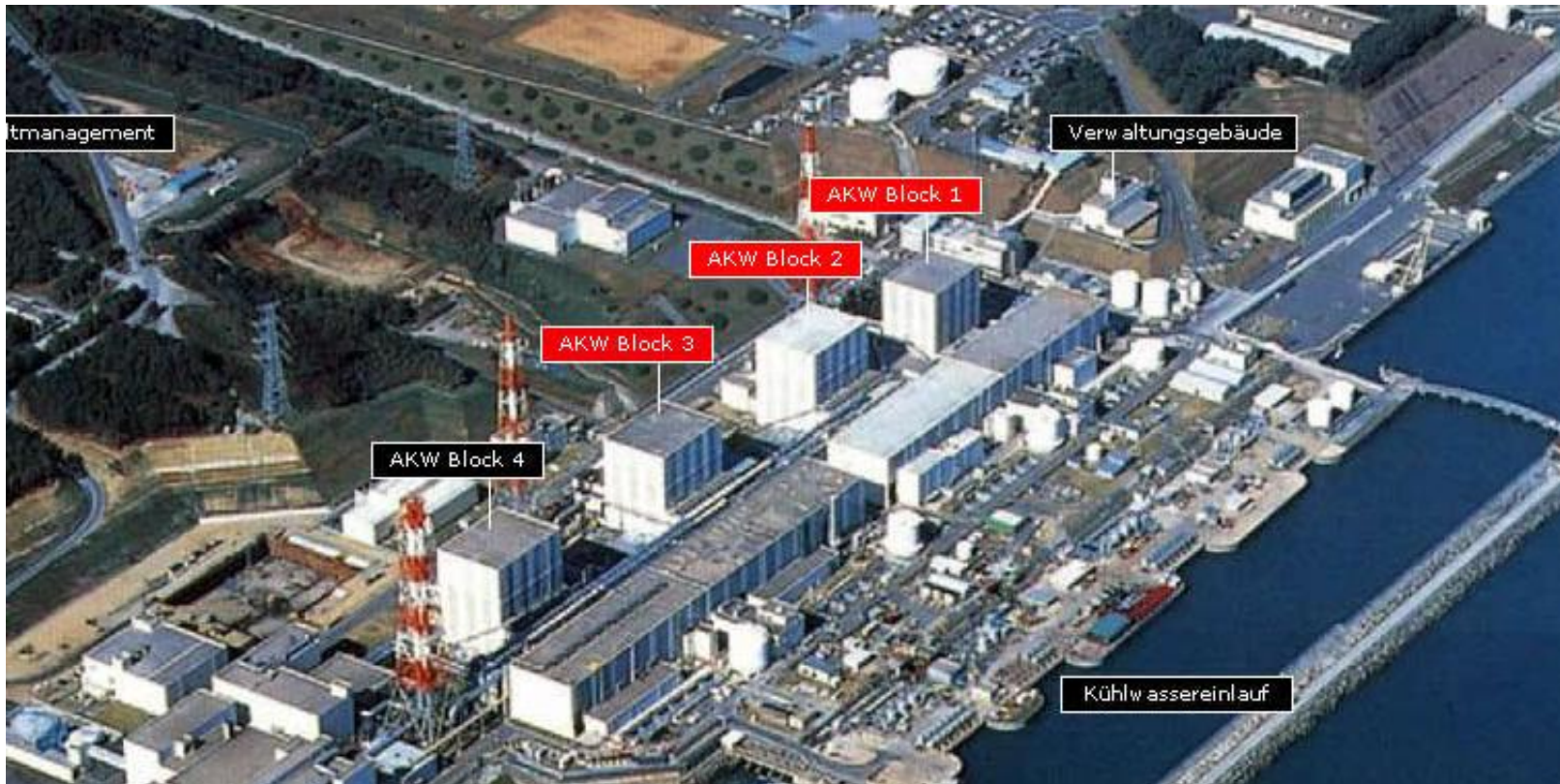
Matthias Braun
PEPA4-G, AREVA–NP GmbH
Matthias.Braun@AREVA.com

The Fukushima Daiichi Incident

1. Plant Design

► Fukushima Daiichi (Plant I)

- ◆ Unit I - GE Mark I BWR (439 MW), Operating since 1971
- ◆ Unit II-IV - GE Mark I BWR (760 MW), Operating since 1974

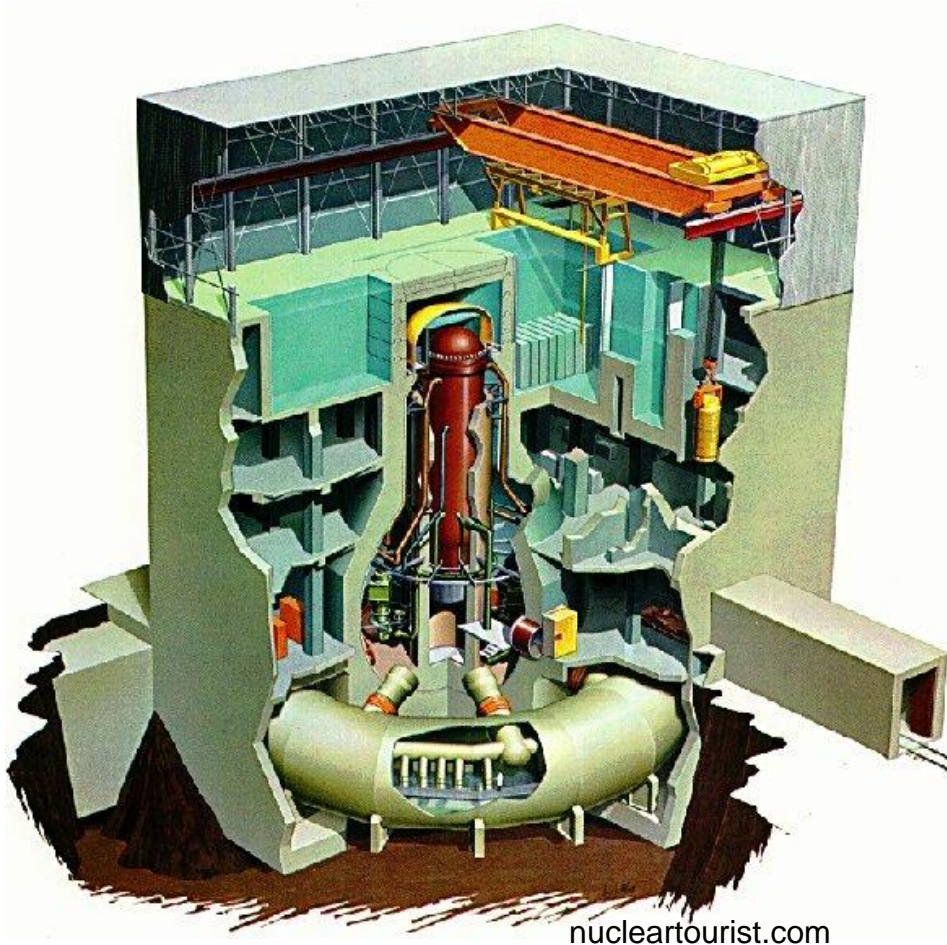


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1. Plant Design

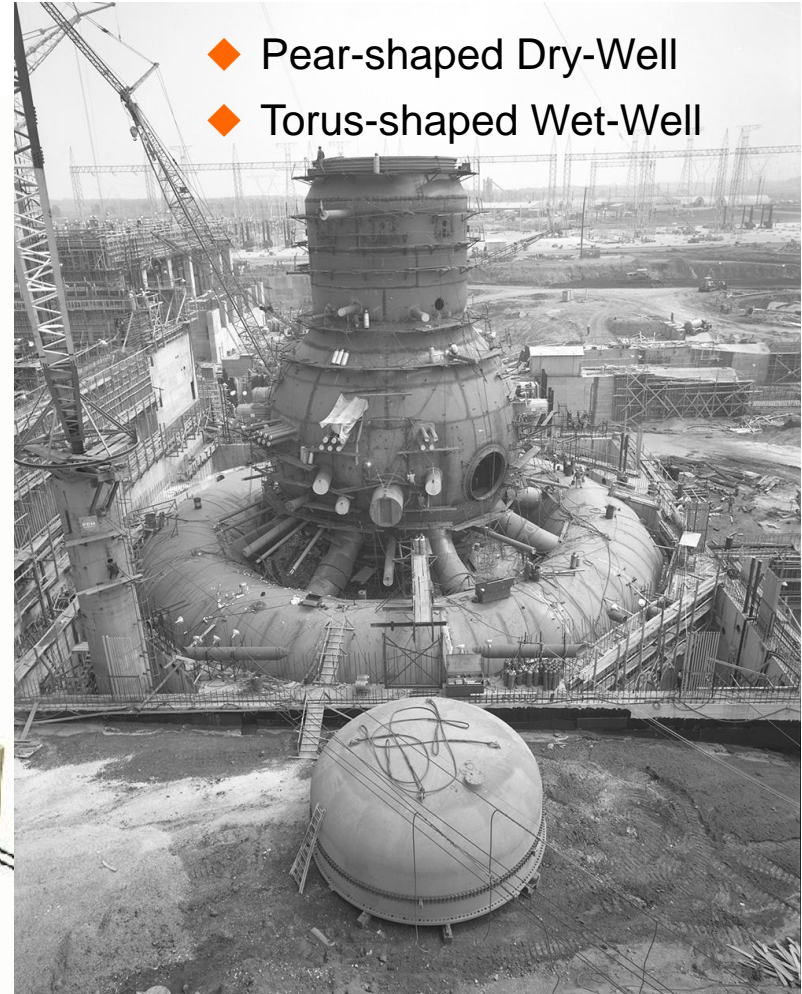
► Building structure

- ◆ Concrete Building
- ◆ Steel-framed Service Floor



► Containment

- ◆ Pear-shaped Dry-Well
- ◆ Torus-shaped Wet-Well



en.wikipedia.org/wiki/Browns_Ferry_Nuclear_Power_Plant

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1. Plant Design

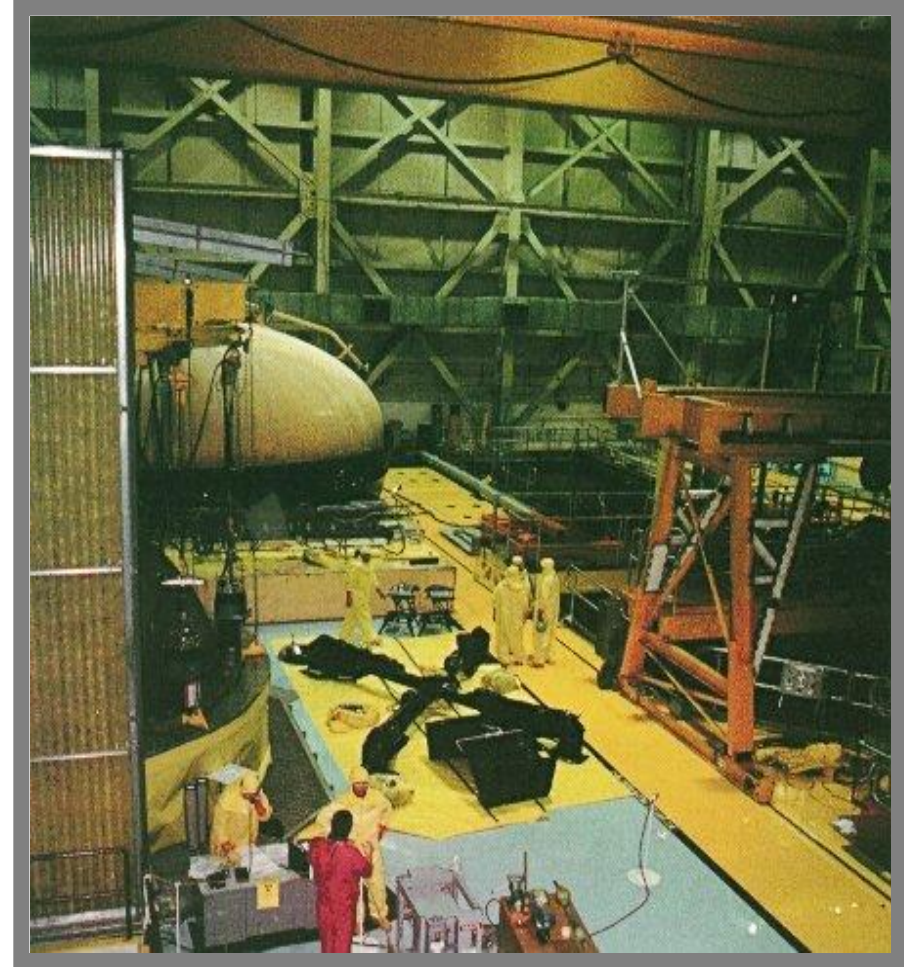
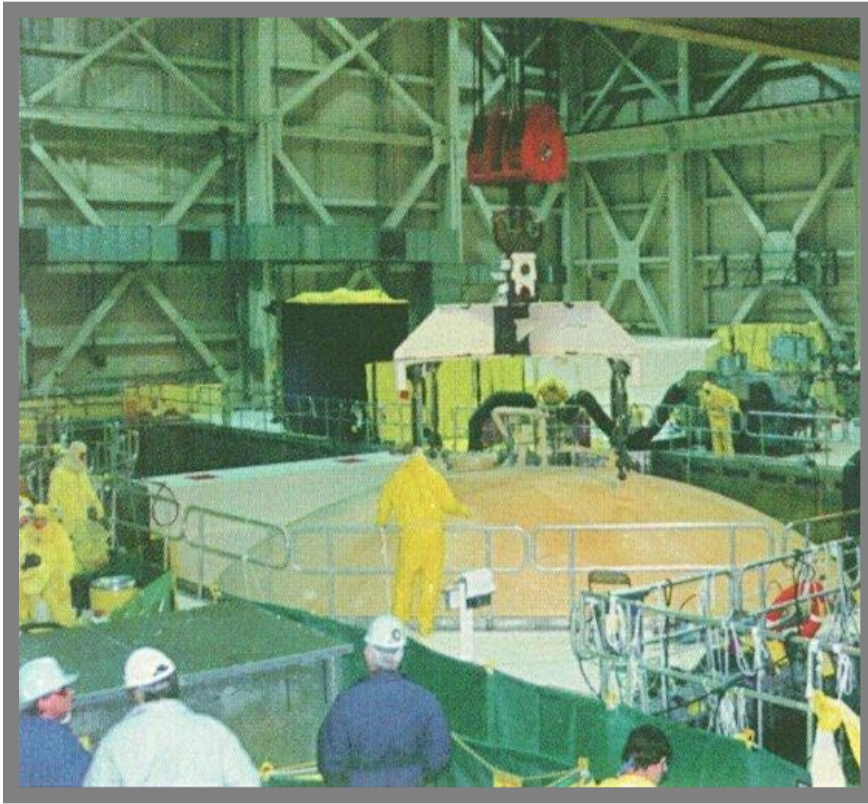
► Service Floor



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1. Plant Design

- ▶ Lifting the Containment closure head



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1. Plant Design

▶ Reactor Service Floor
(Steel Construction)

▶ Concrete Reactor Building
(secondary Containment)

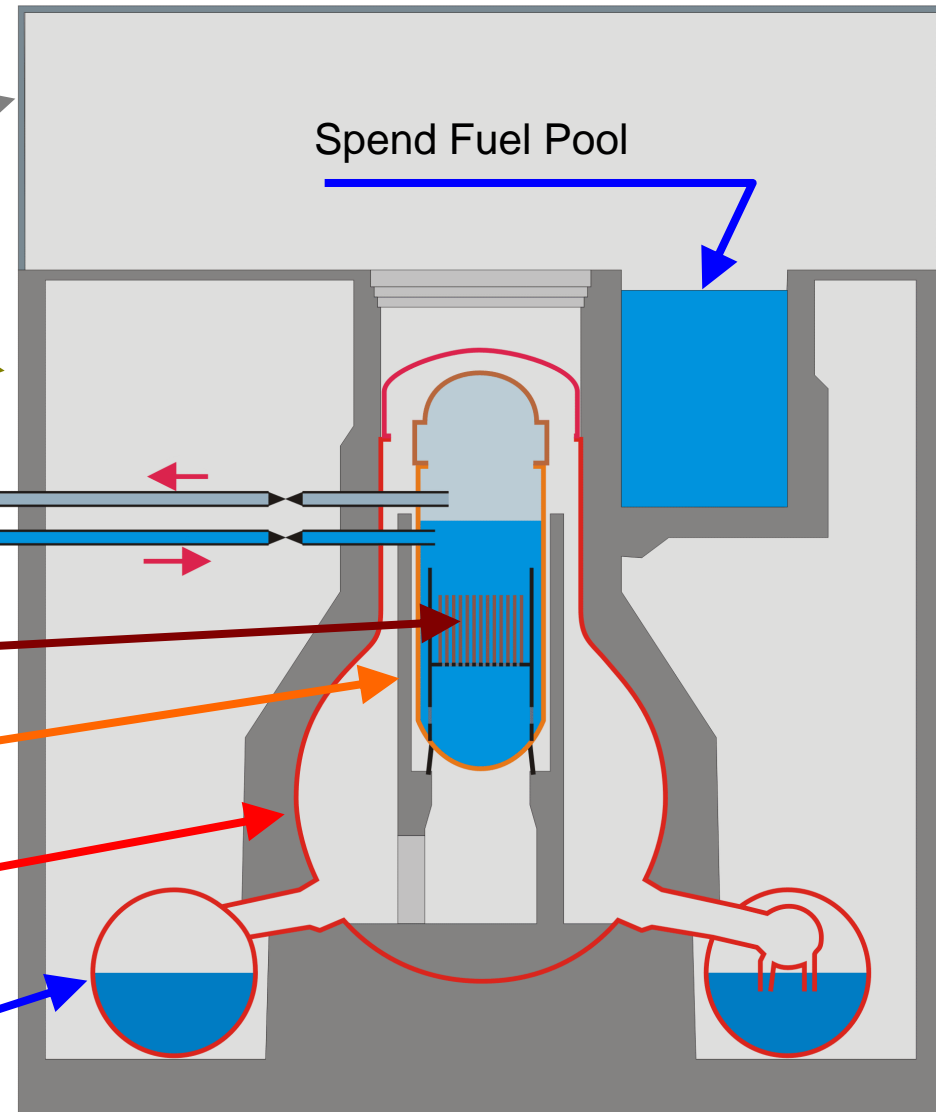
Fresh Steam line
Main Feedwater

▶ Reactor Core

▶ Reactor Pressure Vessel

▶ Containment (Dry well)

▶ Containment (Wet Well) /
Condensation Chamber

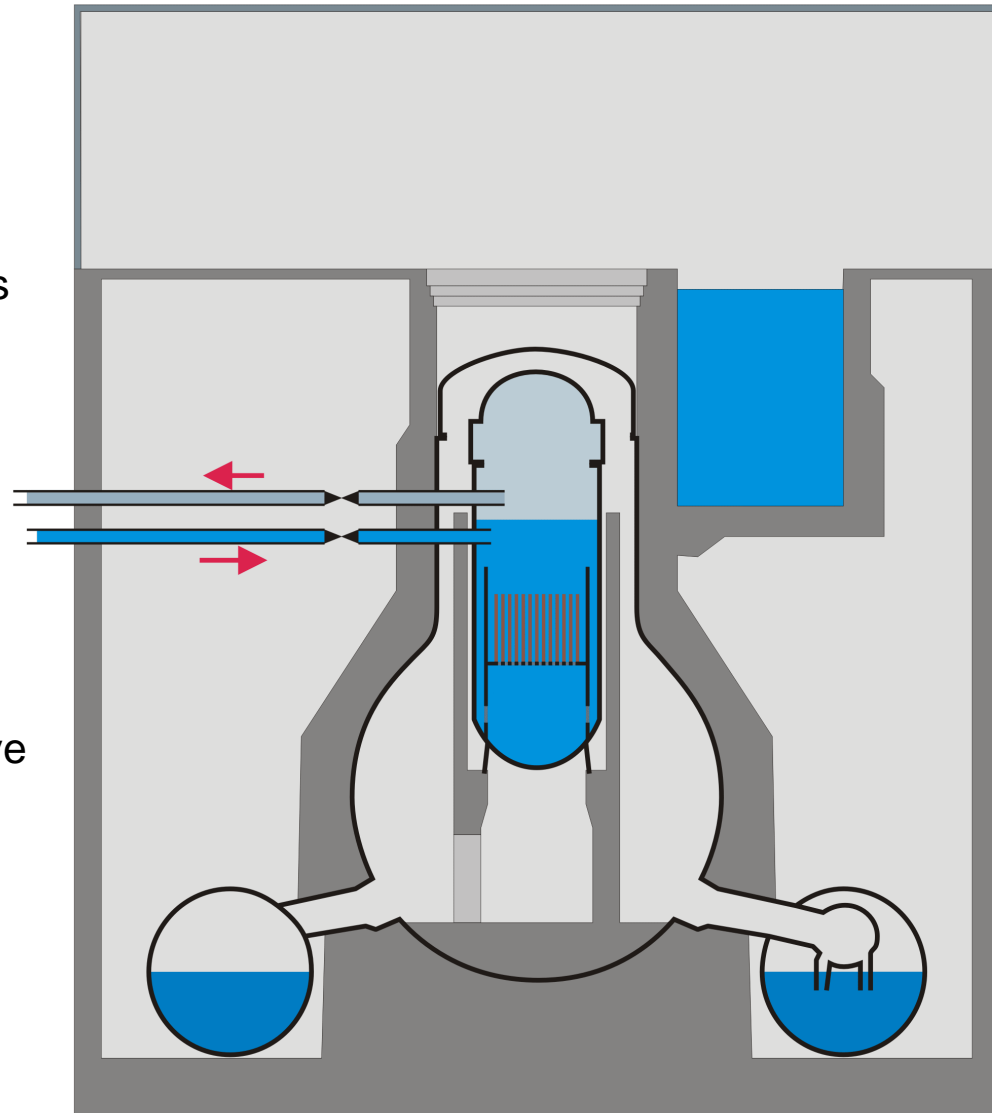


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2. Accident progression

- ▶ 11.3.2011 14:46 - Earthquake
 - ◆ Magnitude 9
 - ◆ Power grid in northern Japan fails
 - ◆ Reactors itself are mainly undamaged

- ▶ SCRAM
 - ◆ Power generation due to Fission of Uranium stops
 - ◆ Heat generation due to radioactive Decay of Fission Products
 - After Scram ~6%
 - After 1 Day ~1%
 - After 5 Days ~0.5%



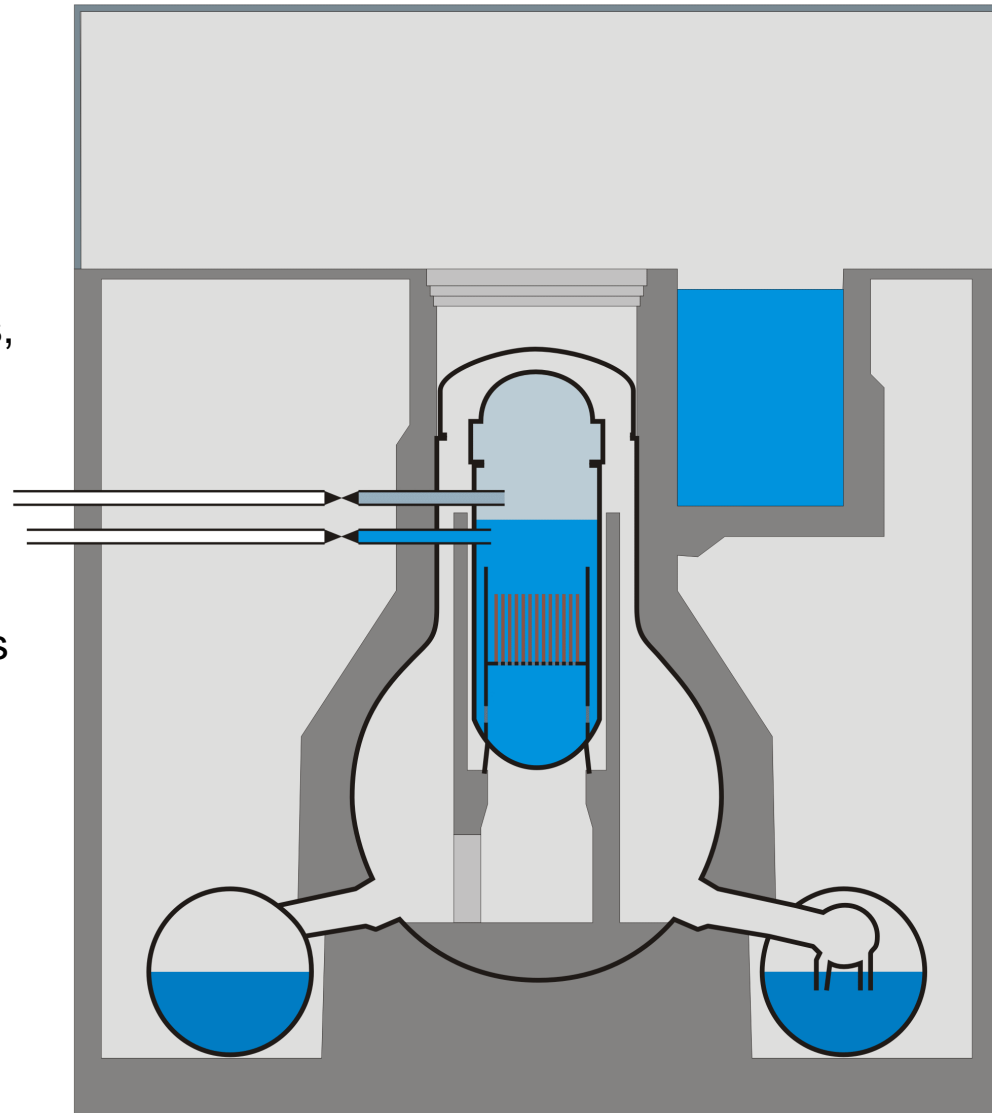
The Fukushima Daiichi Incident

2. Accident progression

- ▶ Containment Isolation
 - ◆ Closing of all non-safety related Penetrations of the containment
 - ◆ Cuts off Machine hall
 - ◆ If containment isolation succeeds, a large early release of fission products is highly unlikely

- ▶ Diesel generators start
 - ◆ Emergency Core cooling systems are supplied

- ▶ Plant is in a stable save state

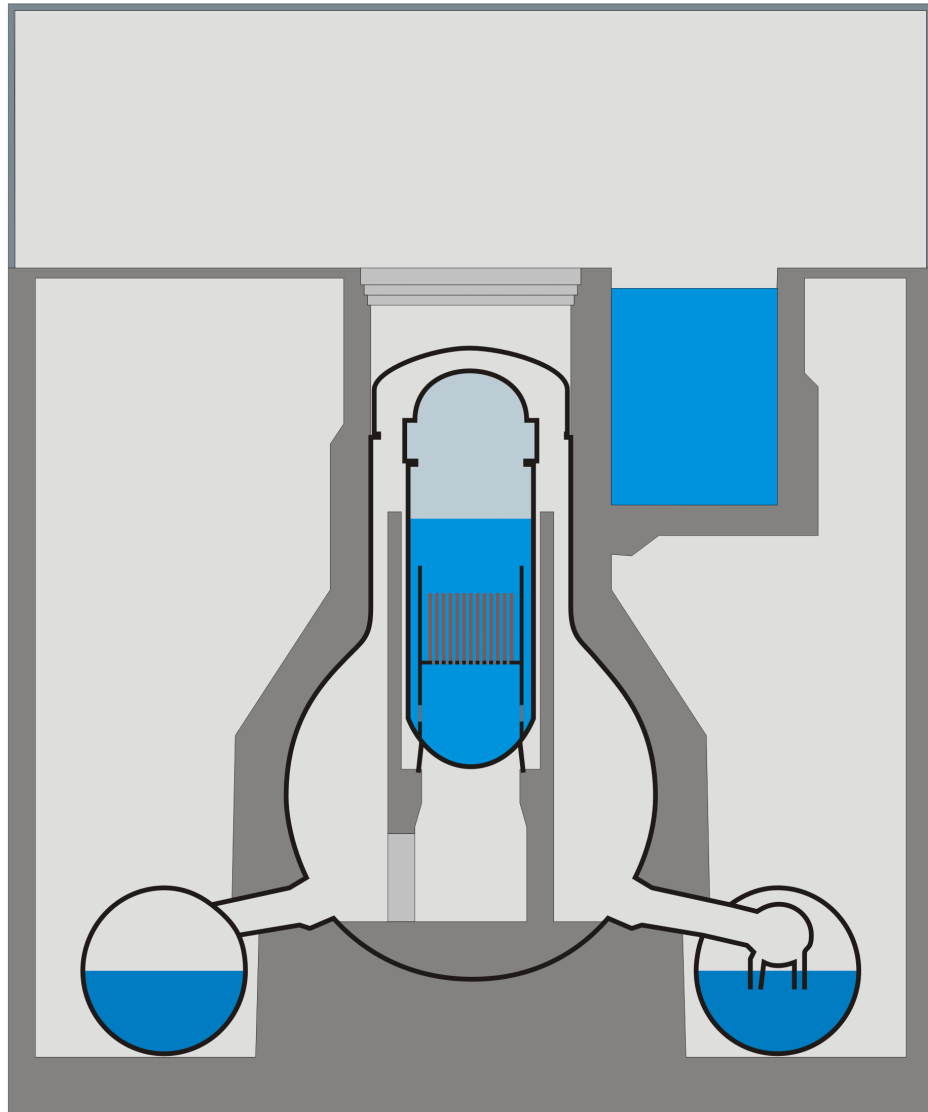


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2. Accident progression

- ▶ 11.3. 15:41 Tsunami hits the plant
 - ◆ Plant Design for Tsunami height of up to 6.5m
 - ◆ Actual Tsunami height >7m
 - ◆ Flooding of
 - Diesel Generators and/or
 - Essential service water building cooling the generators

- ▶ Station Blackout
 - ◆ Common cause failure of the power supply
 - ◆ Only Batteries are still available
 - ◆ Failure of all but one Emergency core cooling systems

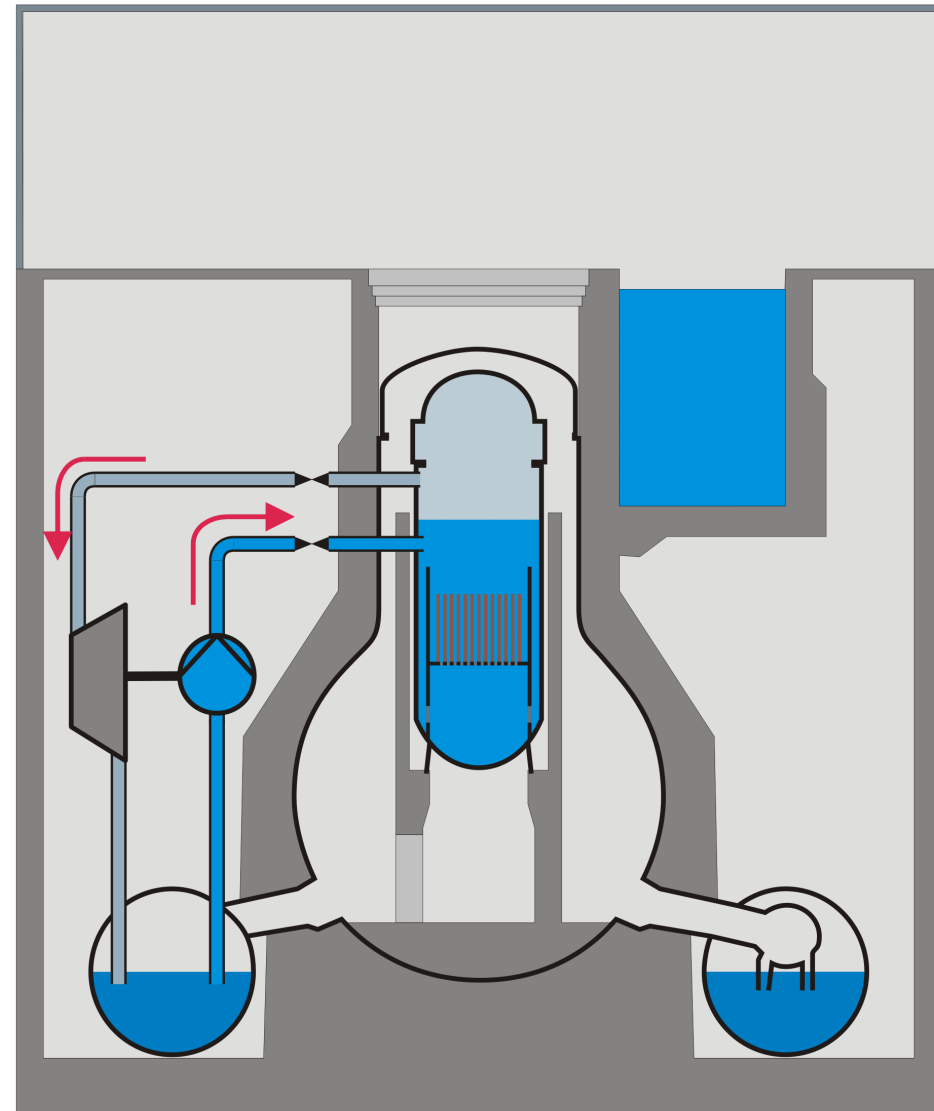


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2. Accident progression

- ▶ Reactor Core Isolation Pump still available
 - ◆ Steam from the Reactor drives a Turbine
 - ◆ Steam gets condensed in the Wet-Well
 - ◆ Turbine drives a Pump
 - ◆ Water from the Wet-Well gets pumped in Reactor
 - ◆ Necessary:
 - Battery power
 - Temperature in the wet-well must be below 100°C

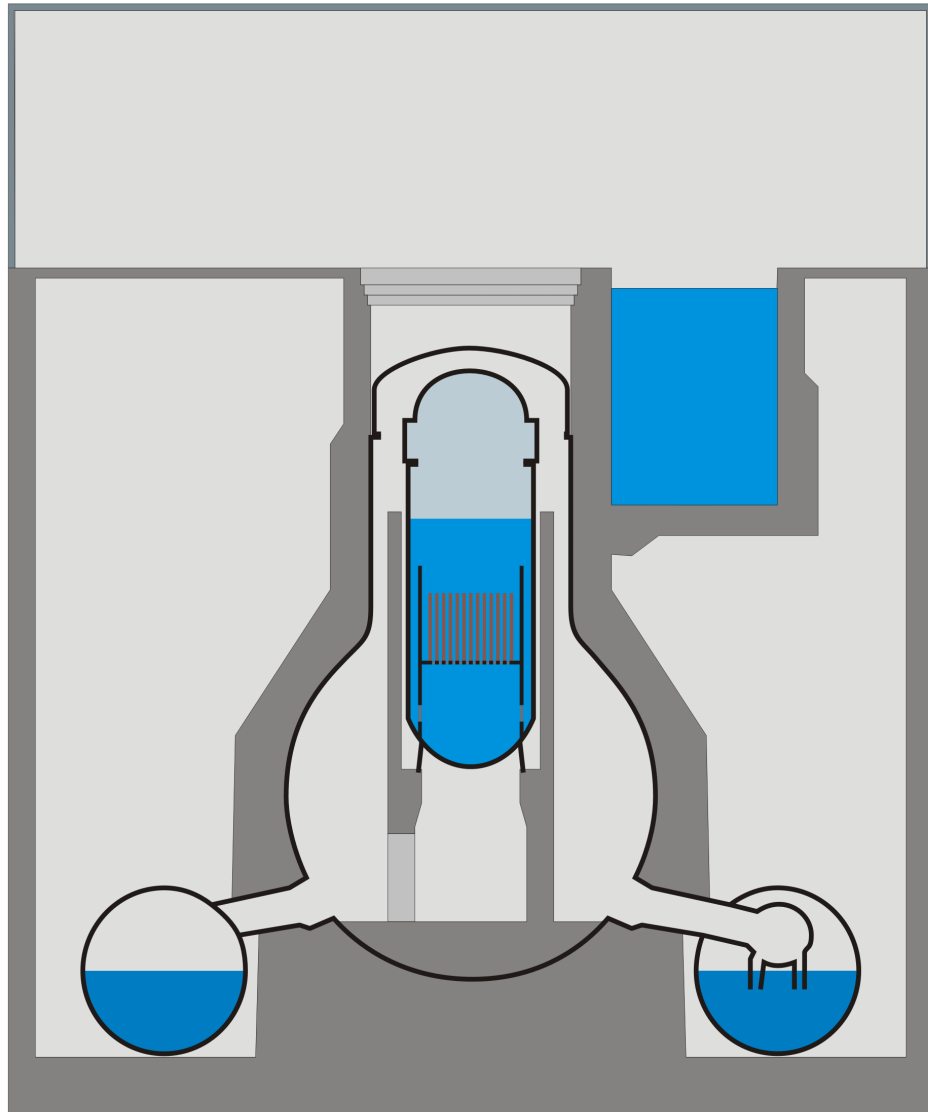
- ▶ As there is no heat removal from the building, the Core isolation pump cant work infinitely



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2. Accident progression

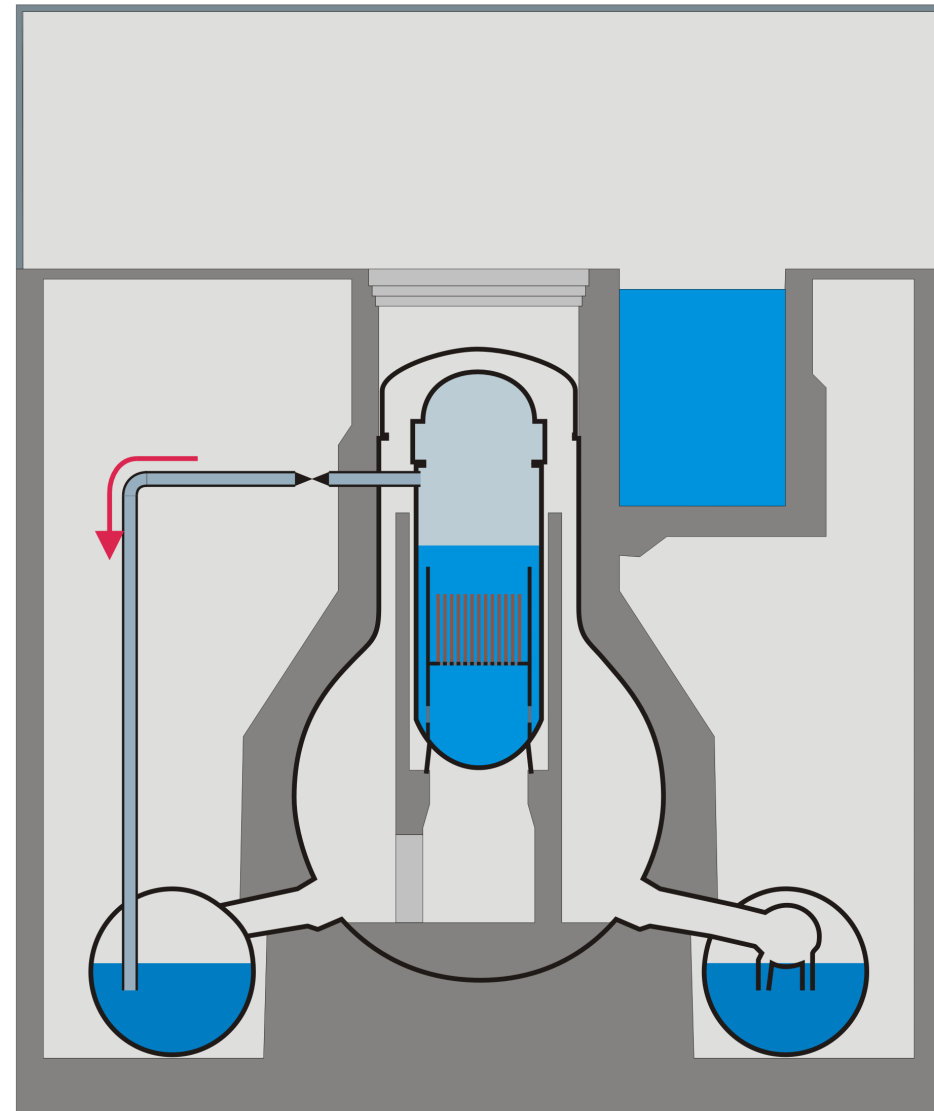
- ▶ Reactor Isolation pump stops
 - ◆ 11.3. 16:36 in Unit 1 (Batteries empty)
 - ◆ 14.3. 13:25 in Unit 2 (Pump failure)
 - ◆ 13.3. 2:44 in Unit 3 (Batteries empty)
- ▶ Decay Heat produces still steam in Reactor pressure Vessel
 - ◆ Pressure rising
- ▶ Opening the steam relieve valves
 - ◆ Discharge Steam into the Wet-Well
- ▶ Descending of the Liquid Level in the Reactor pressure vessel



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2. Accident progression

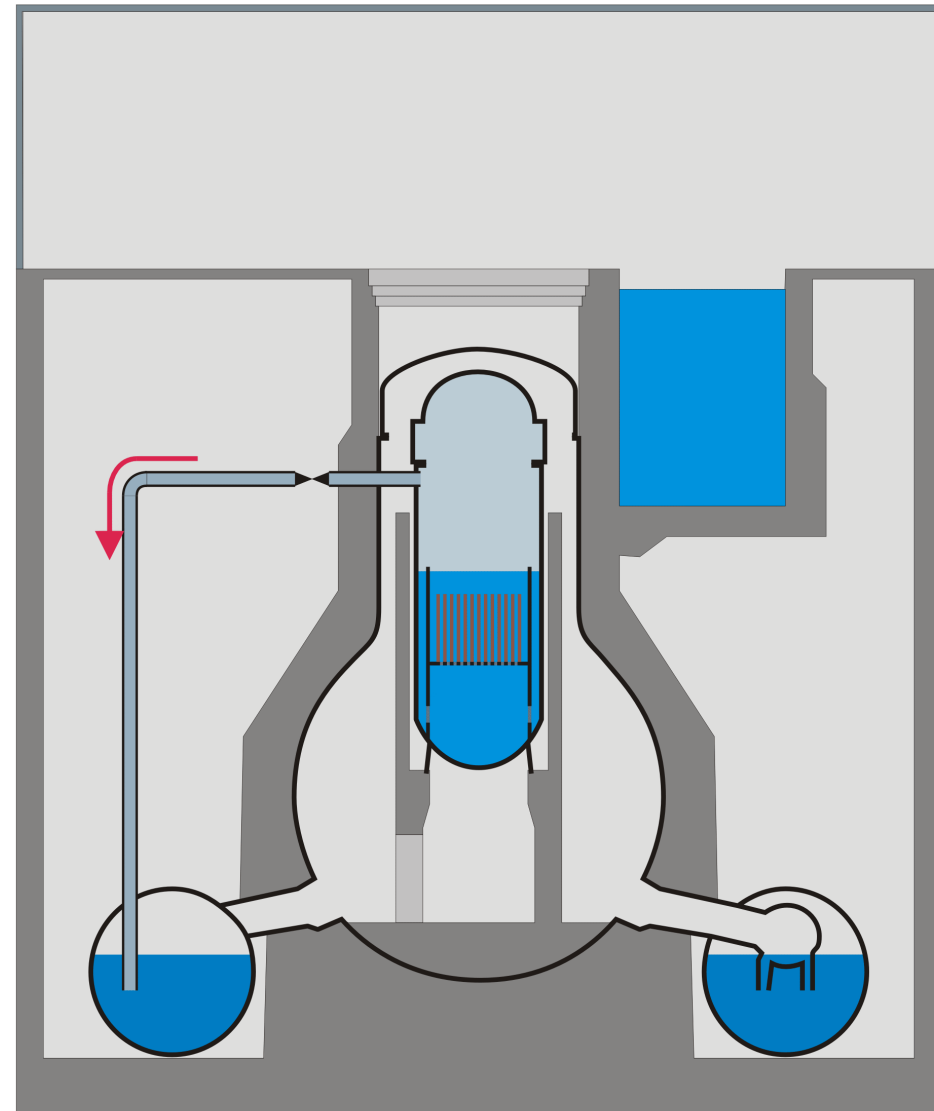
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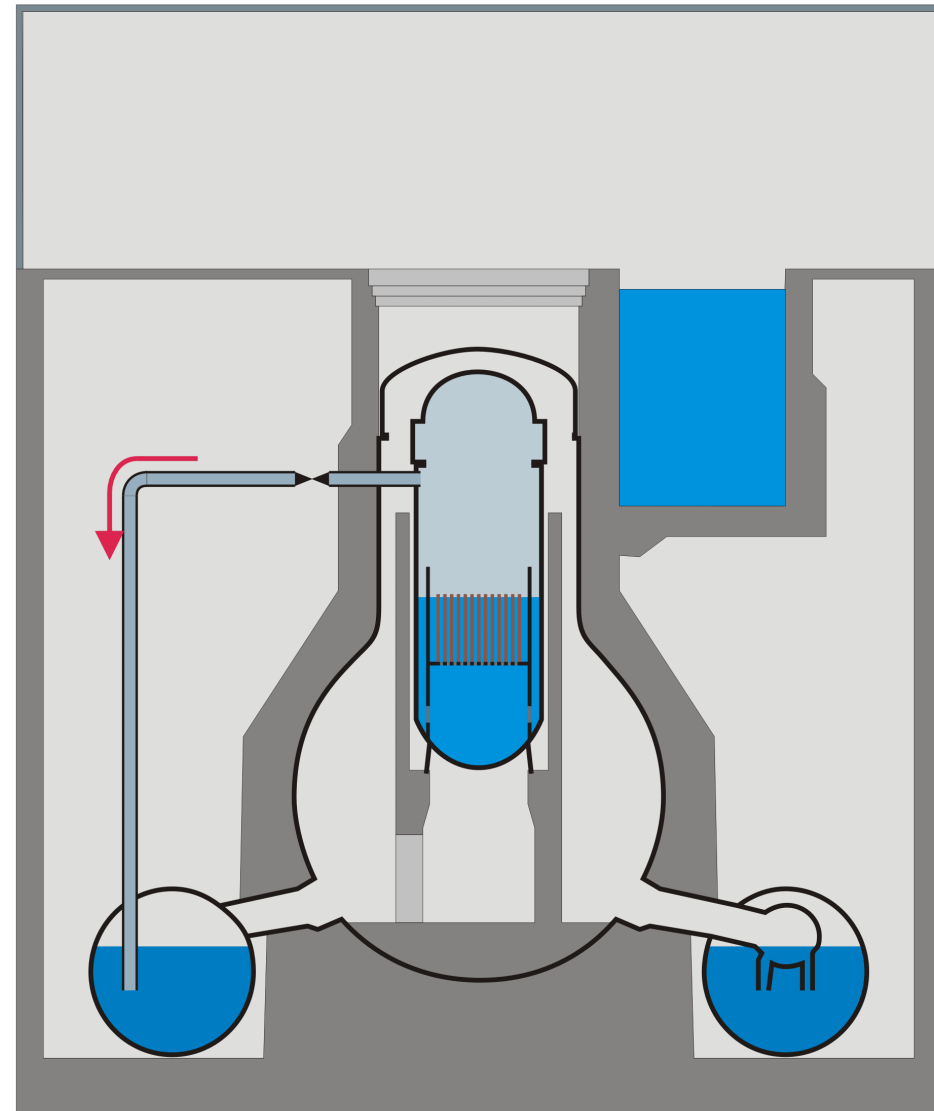
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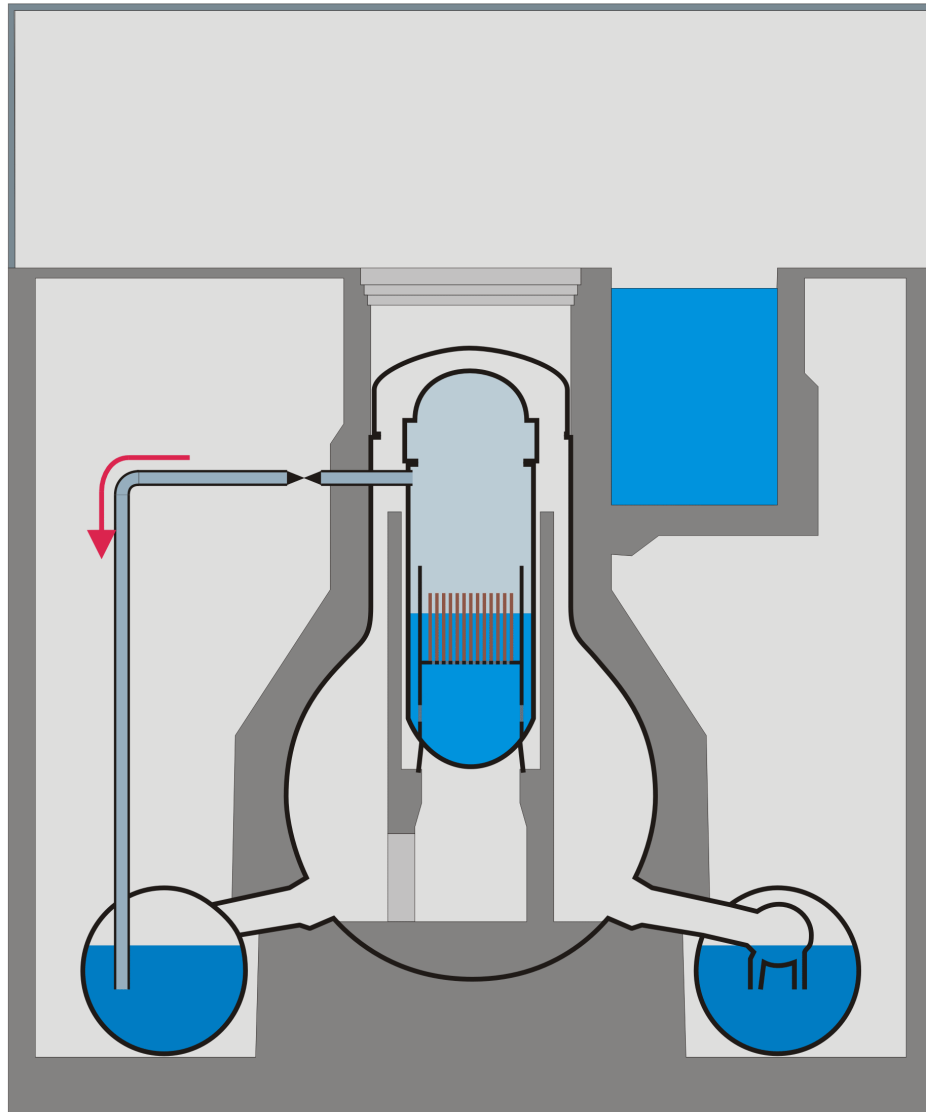
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The Fukushima Daiichi Incident

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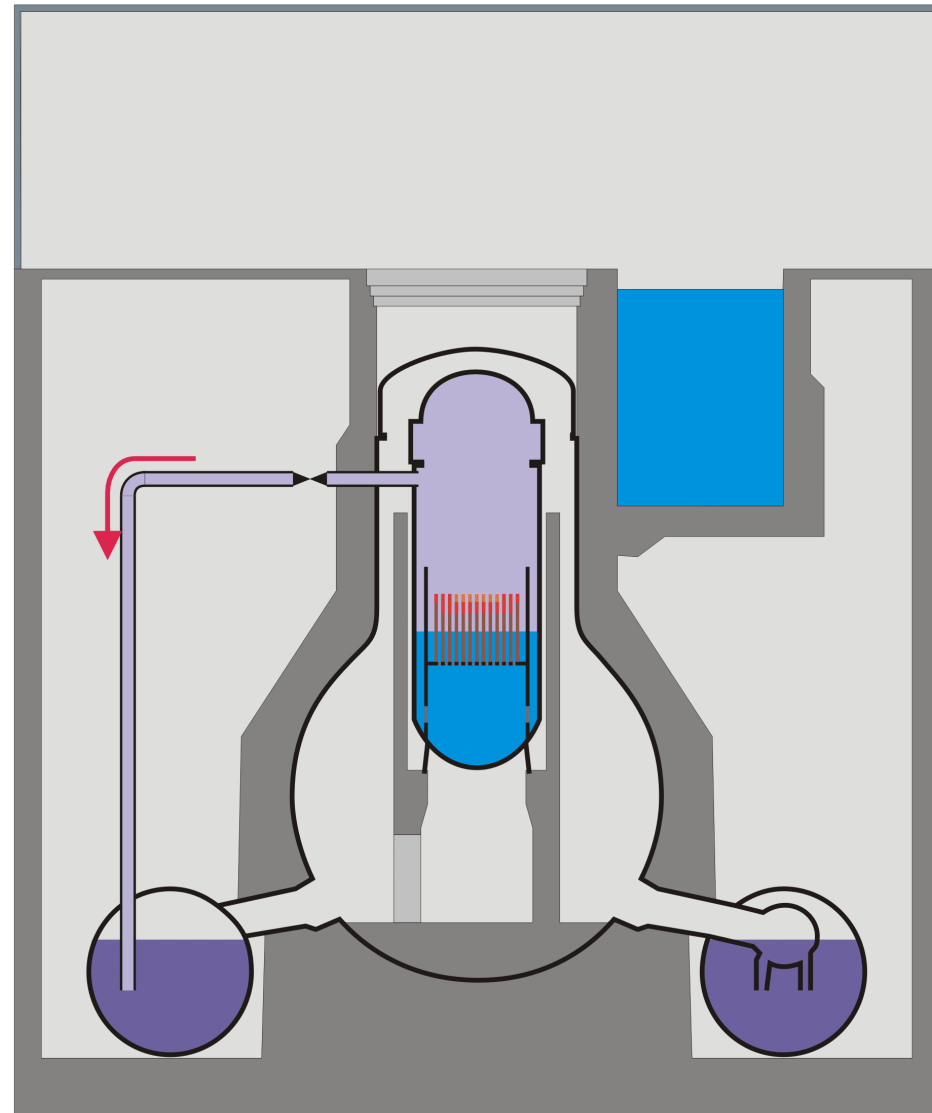
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2. Accident progression

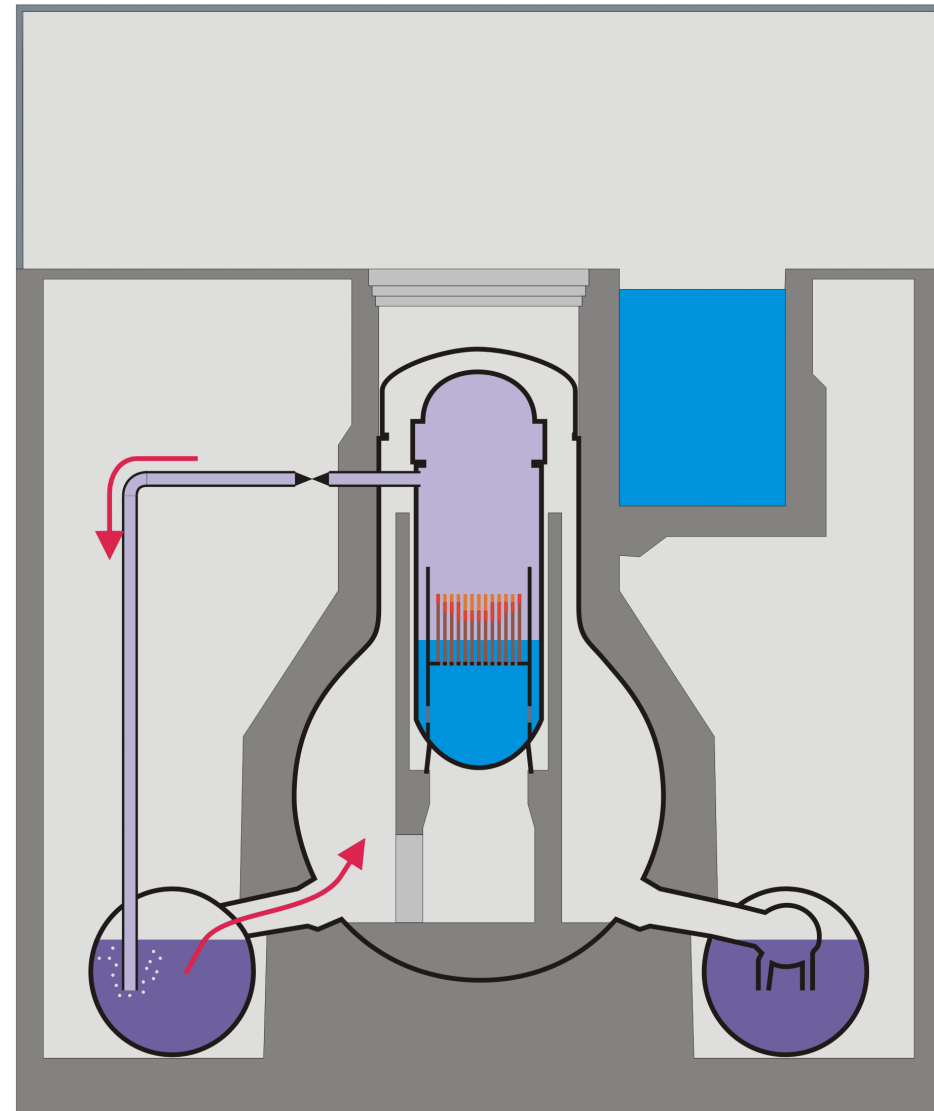
- ▶ Measured, and here referenced Liquid level is the collapsed level. The actual liquid level lies higher due to the steam bubbles in the liquid
- ▶ ~50% of the core exposed
 - ◆ Cladding temperatures rise, but still no significant core damage
- ▶ ~2/3 of the core exposed
 - ◆ Cladding temperature exceeds $\sim 900^{\circ}\text{C}$
 - ◆ Ballooning / Breaking of the cladding
 - ◆ Release of fission products from the fuel rod gaps



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2. Accident progression

- ▶ ~3/4 of the core exposed
 - ◆ Cladding exceeds ~1200°C
 - ◆ Zirconium in the cladding starts to burn under Steam atmosphere
 - ◆ $\text{Zr} + 2\text{H}_2\text{O} \rightarrow \text{ZrO}_2 + 2\text{H}_2$
 - ◆ Exothermal reaction further heats the core
 - ◆ Generation of hydrogen
 - Unit 1: 300-600kg
 - Unit 2/3: 300-1000kg
 - ◆ Hydrogen gets pushed via the wet-well, the wet-well vacuum breakers into the dry-well



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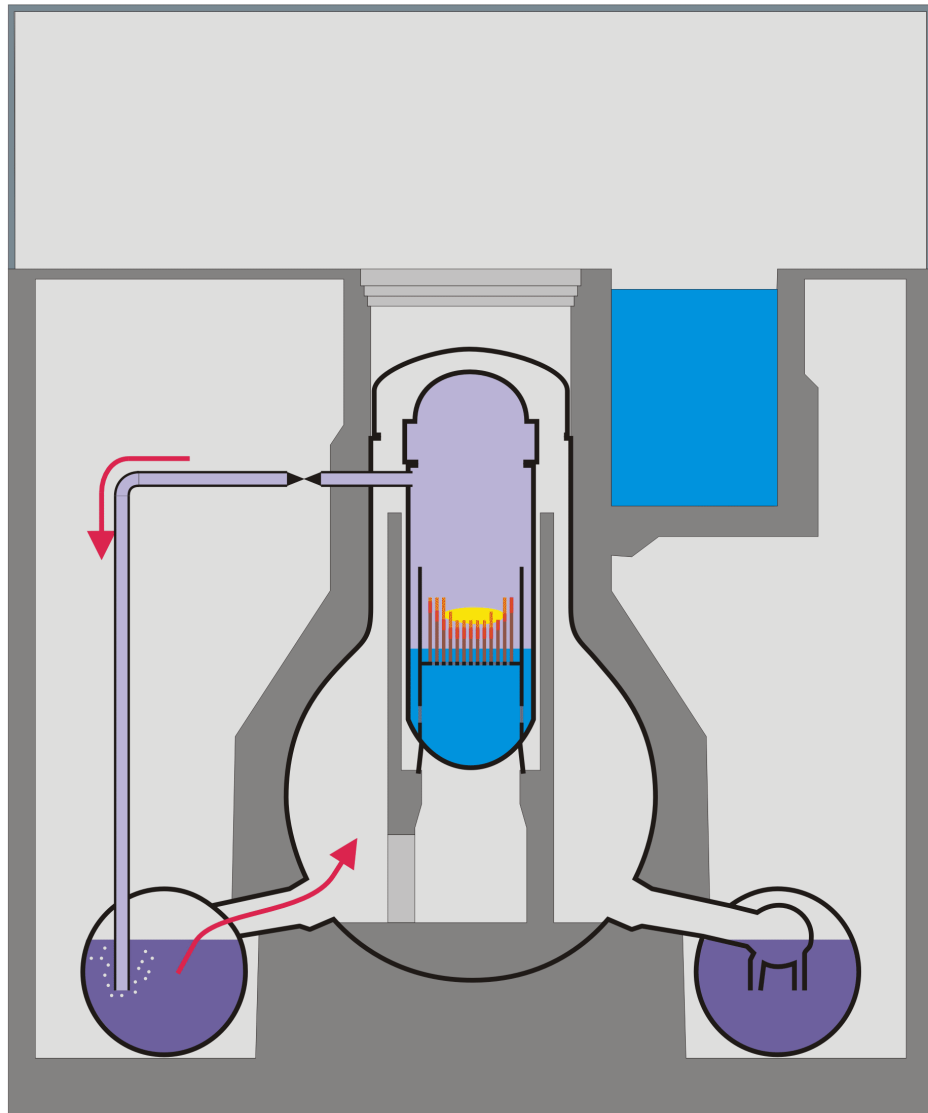
2. Accident progression

- ▶ at ~1800°C [Unit 1,2,3]
 - ◆ Melting of the Cladding
 - ◆ Melting of the steel structures

- ▶ at ~2500°C [Block 1,2]
 - ◆ Breaking of the fuel rods
 - ◆ debris bed inside the core

- ▶ at ~2700°C [Block 1]
 - ◆ Melting of Uranium-Zirconium eutectics

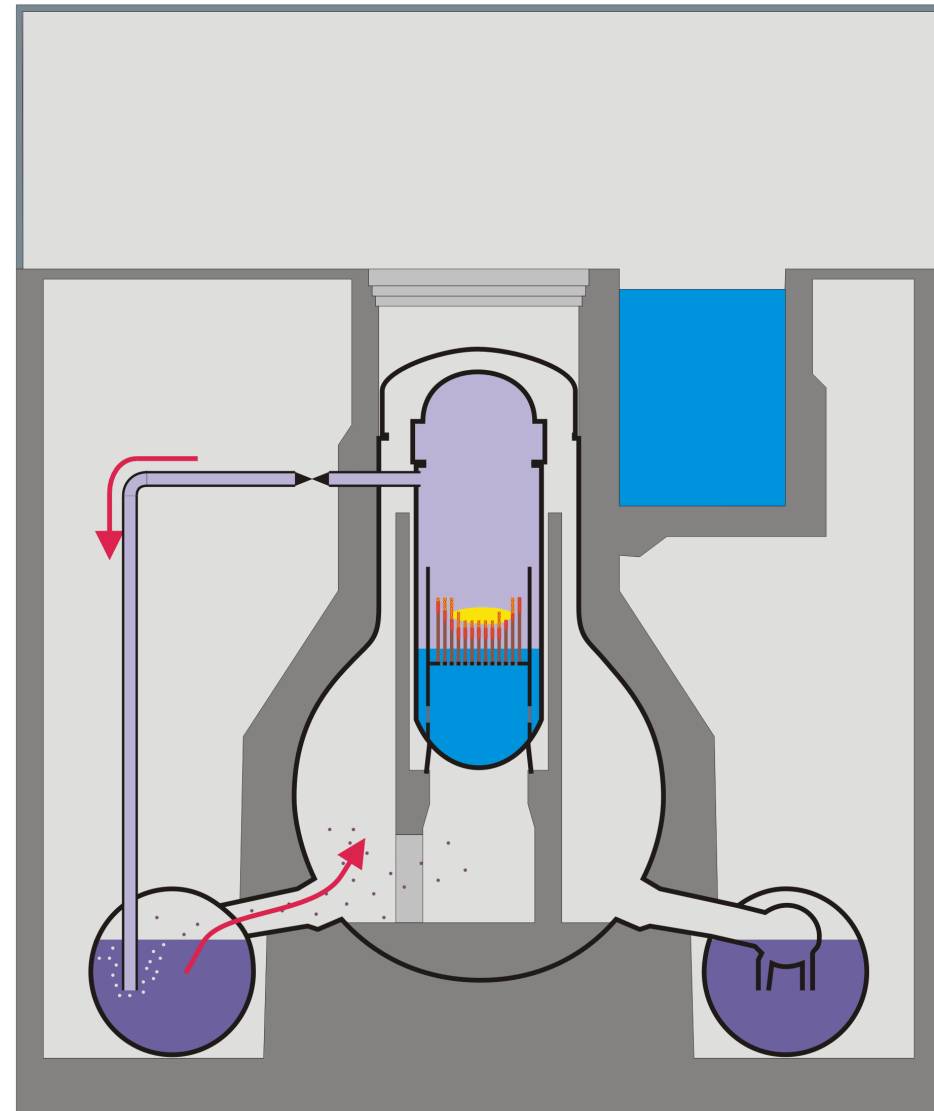
- ▶ Restoration of the water supply stops accident in all 3 Units
 - ◆ Unit 1: 12.3. 20:20 (27h w.o. water)
 - ◆ Unit 2: 14.3. 20:33 (7h w.o. water)
 - ◆ Unit 3: 13.3. 9:38 (7h w.o. water)



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2. Accident progression

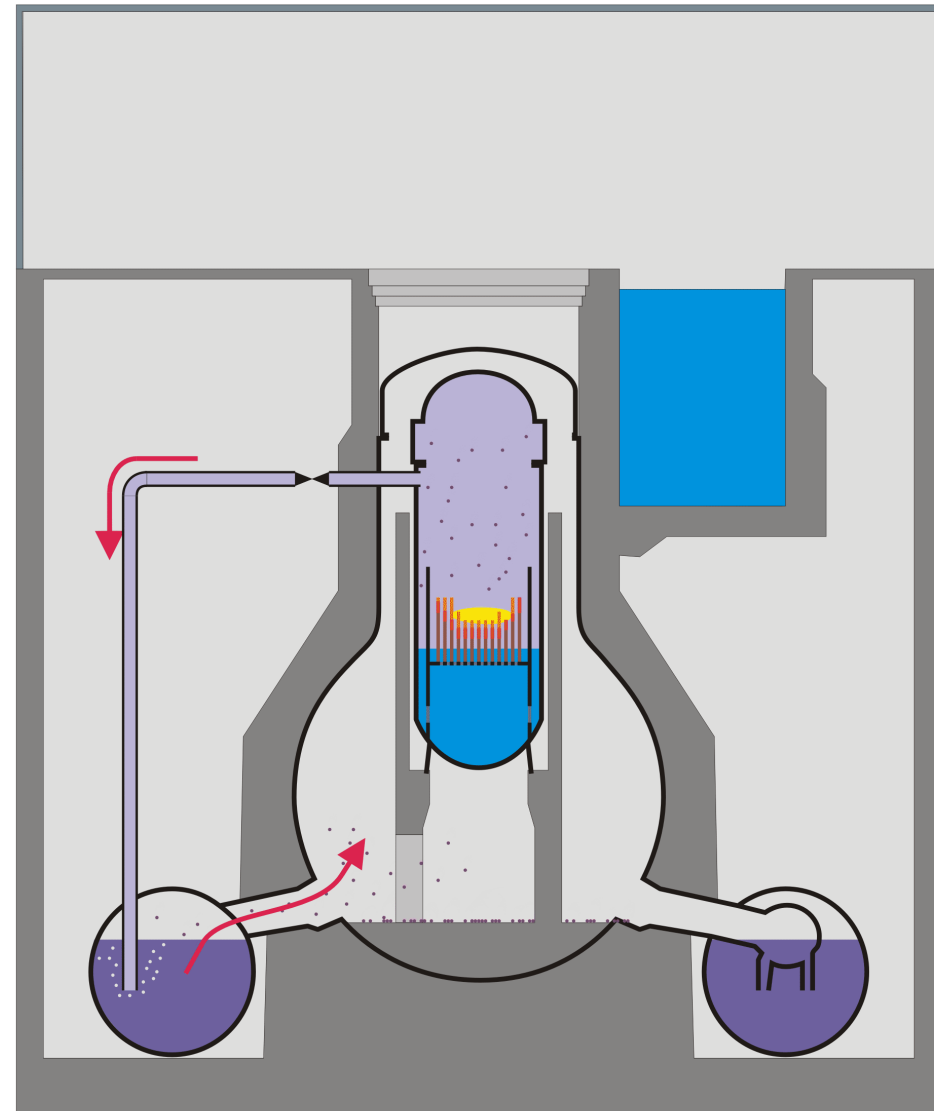
- ▶ Release of fission products during melt down
 - ◆ Xenon, Cesium, Iodine,...
 - ◆ Uranium/Plutonium remain in core
 - ◆ Fission products condensate to airborne Aerosols
- ▶ Discharge through valves into water of the condensation chamber
 - ◆ Pool scrubbing binds a fraction of Aerosols in the water
- ▶ Xenon and remaining aerosols enter the Dry-Well
 - ◆ Deposition of aerosols on surfaces further decontaminates air



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2. Accident progression

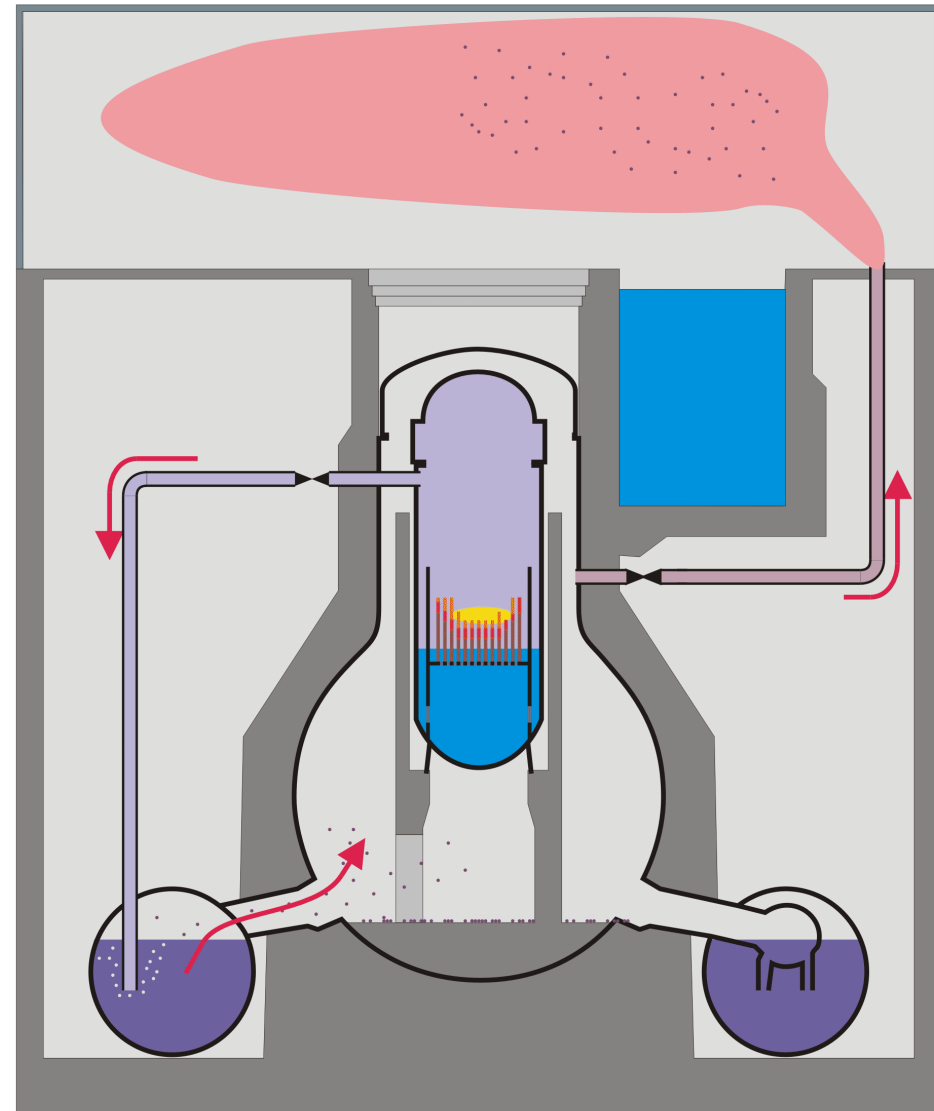
- ▶ Containment
 - ◆ Last barrier between Fission Products and Environment
 - ◆ Wall thickness ~3cm
 - ◆ Design Pressure 4-5bar
- ▶ Actual pressure up to 8 bars
 - ◆ Normal inert gas filling (Nitrogen)
 - ◆ Hydrogen from core oxidation
 - ◆ Boiling condensation chamber (like a pressure cooker)
- ▶ Depressurization of the containment
 - ◆ Unit 1: 12.3. 4:00
 - ◆ Unit 2: 13.3 00:00
 - ◆ Unit 3: 13.3. 8.41



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2. Accident progression

- ▶ Positive und negative Aspects of depressurizing the containment
 - ◆ Removes Energy from the Reactor building (only way left)
 - ◆ Reducing the pressure to ~4 bar
 - ◆ Release of small amounts of Aerosols (Iodine, Cesium ~0.1%)
 - ◆ Release of all noble gases
 - ◆ Release of Hydrogen
- ▶ Gas is released into the reactor service floor
 - ◆ Hydrogen is flammable

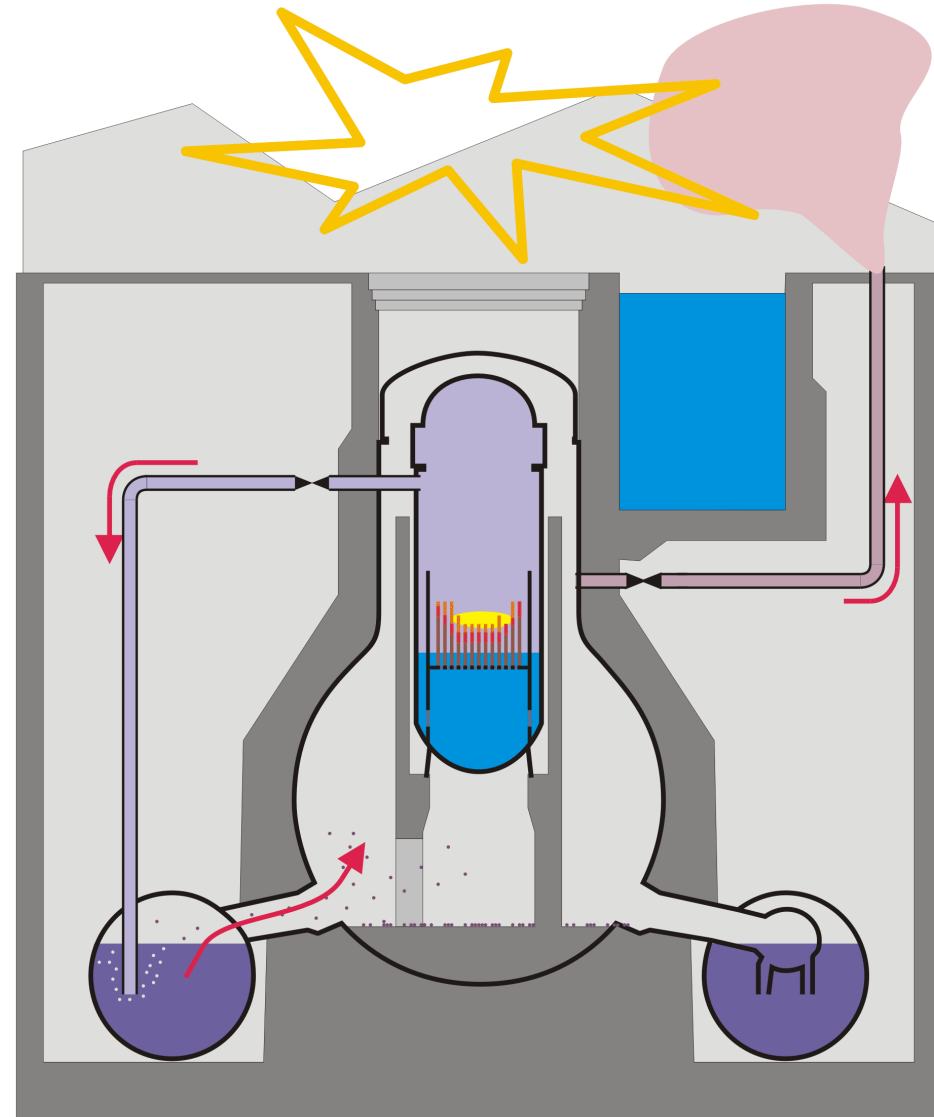


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2. Accident progression

▶ Unit 1 und 3

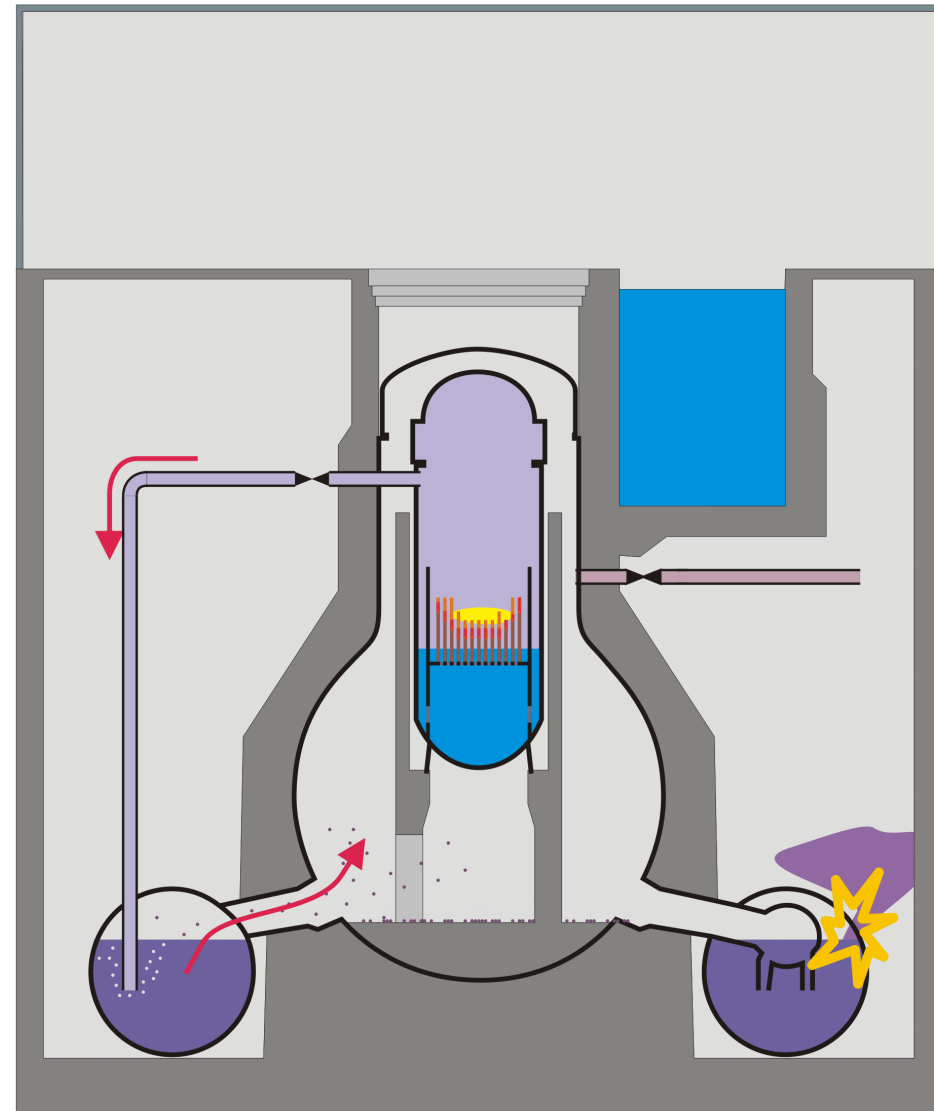
- ◆ Hydrogen burn inside the reactor service floor
- ◆ Destruction of the steel-frame roof
- ◆ Reinforced concrete reactor building seems undamaged
- ◆ Spectacular but minor safety relevant



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2. Accident progression

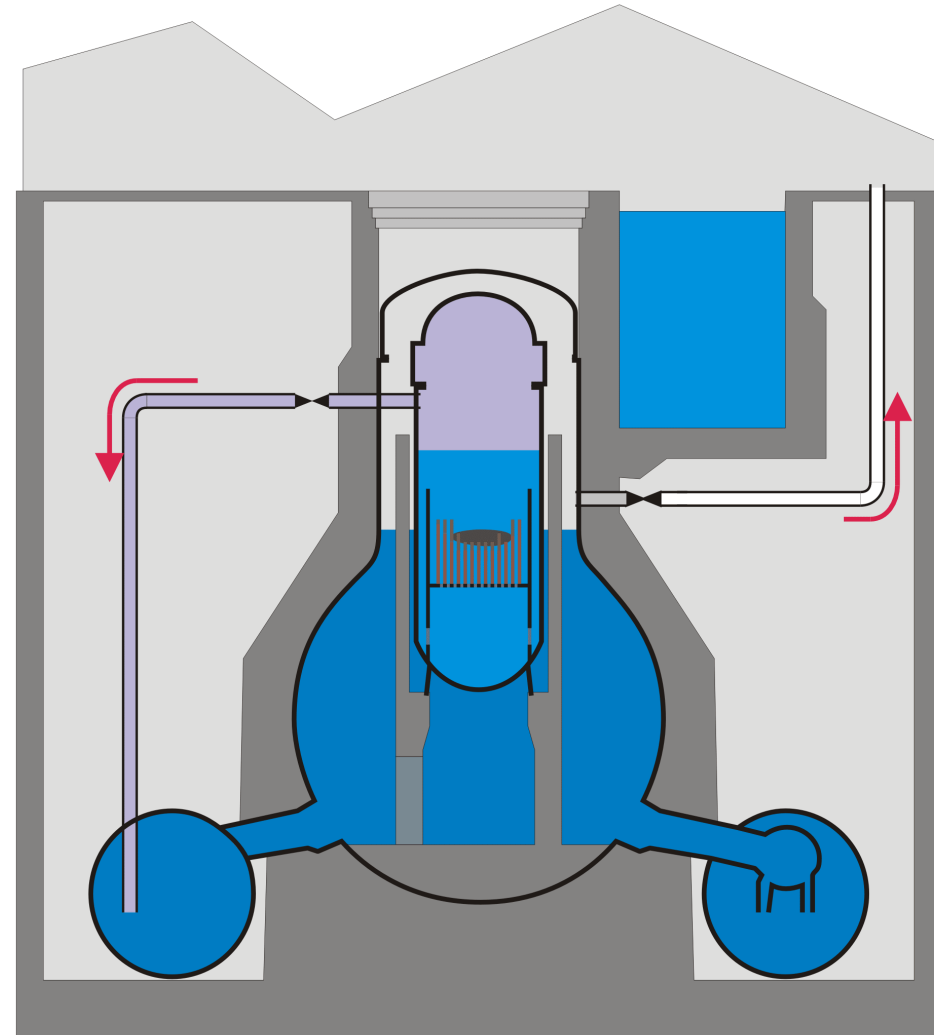
- ▶ Unit 2
 - ◆ Hydrogen burn inside the reactor building
 - ◆ Probably damage to the condensation chamber (highly contaminated water)
 - ◆ Uncontrolled release of gas from the containment
 - ◆ **Release of fission products**
 - ◆ Temporal evacuation of the plant
 - ◆ High local dose rates on the plant site due to wreckage hinder further recovery work
- ▶ No clear information's why Unit 2 behaved differently



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2. Accident progression

- ▶ Current status of the Reactors
 - ◆ Core Damage in Unit 1,2, 3
 - ◆ Building damage due to various burns Unit 1-4
 - ◆ Reactor pressure vessels flooded in all Units with mobile pumps
 - ◆ At least containment in Unit 1 flooded
- ▶ Further cooling of the Reactors by releasing steam to the atmosphere
- ▶ Only small further releases of fission products can be expected



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3. Radiological releases

▶ Directly on the plant site

◆ Before Explosion in Unit Block 2

- Below 2mSv / h
- Mainly due to released radioactive noble gases
- Measuring posts on west side. Maybe too small values measured due to wind

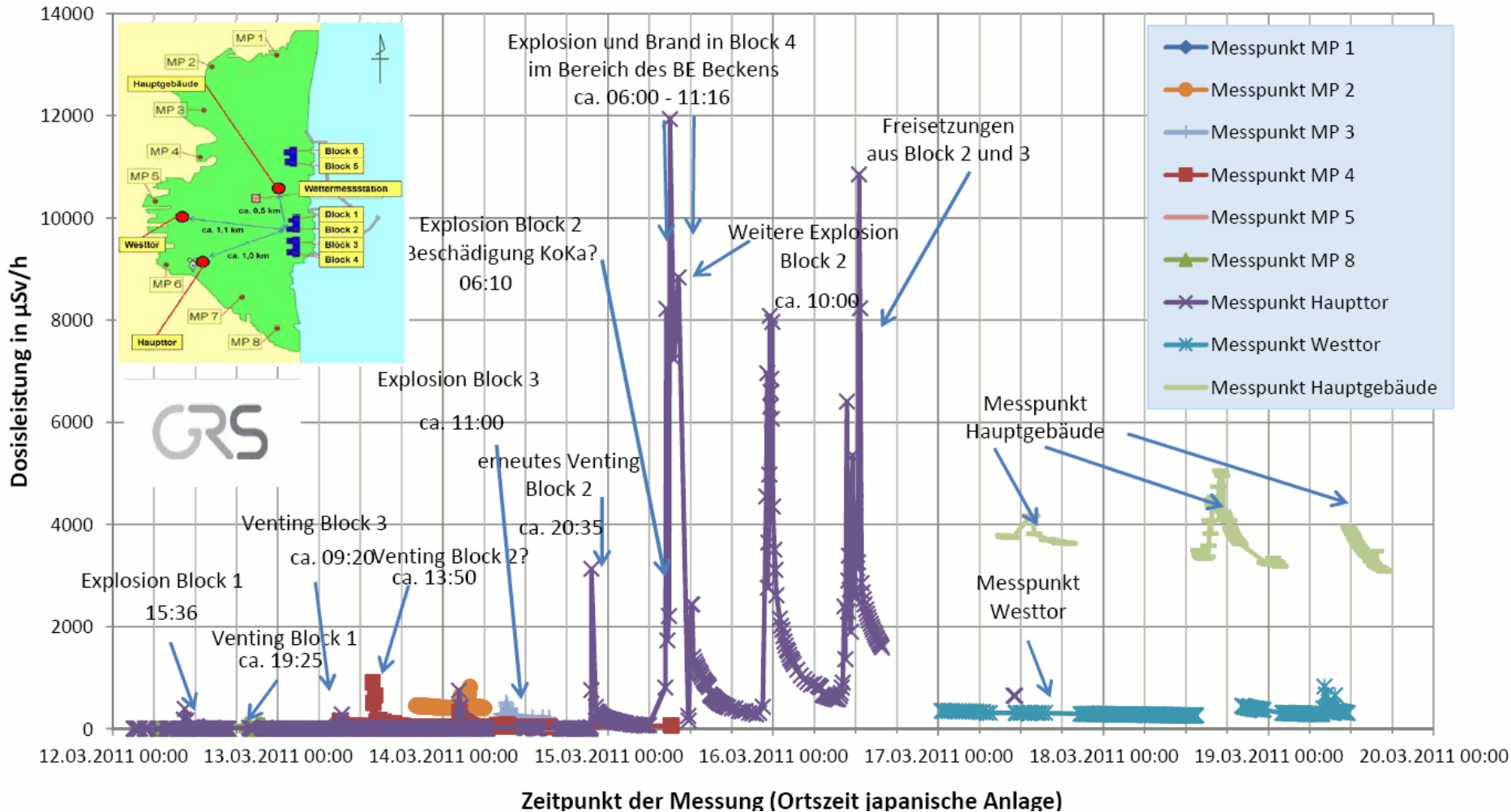
◆ After Explosion in Unit 2 (Damage of the Containment)

- Temporal peak values 12mSv / h
- (Origin not entirely clear)
- Local peak values on site up to 400mSv /h (wreckage / fragments?)
- Currently stable dose on site at 5mSv /h
- Inside the buildings a lot more

◆ Limiting time of exposure of the workers necessary

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3. Radiological releases



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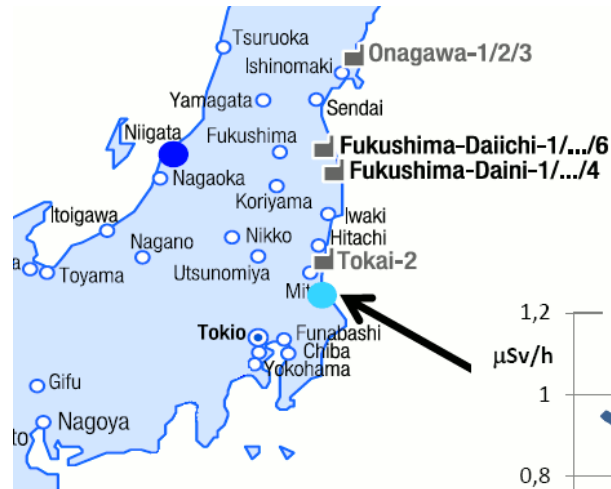
3. Radiological releases

- ▶ Outside the Plant site
 - ◆ As reactor building mostly intact
 - => reduced release of Aerosols (not Chernobyl-like)
 - ◆ Fission product release in steam
 - => fast Aerosol grows, large fraction falls down in the proximity of the plant
 - ◆ Main contribution to the radioactive dose outside plant are the radioactive noble gases
 - ◆ Carried / distributed by the wind, decreasing dose with time
 - ◆ No „Fall-out“ of the noble gases, so no local high contamination of soil

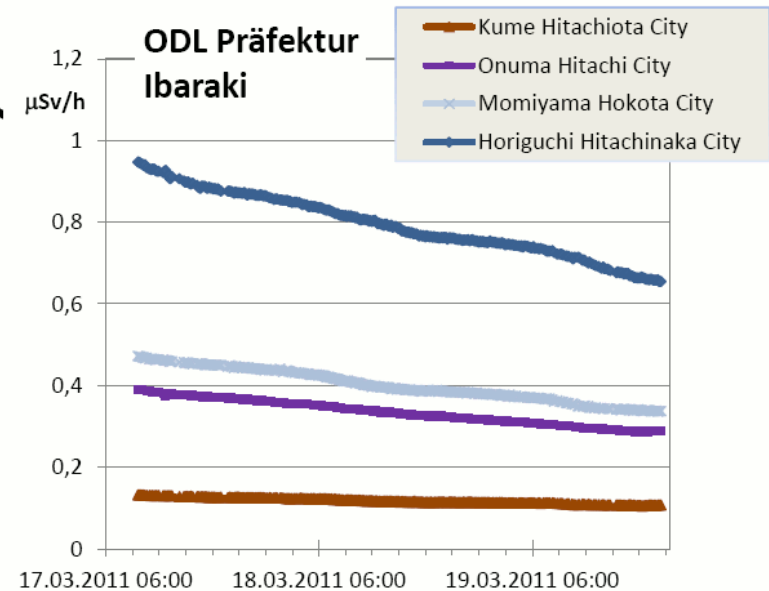
- ▶ ~20km around the plant
 - ◆ Evacuations were adequate
 - ◆ Measured dose up to 0.3mSv/h for short times
 - ◆ Maybe destruction of crops / dairy products this year
 - ◆ Probably no permanent evacuation of land necessary

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3. Radiological releases



GRS.de



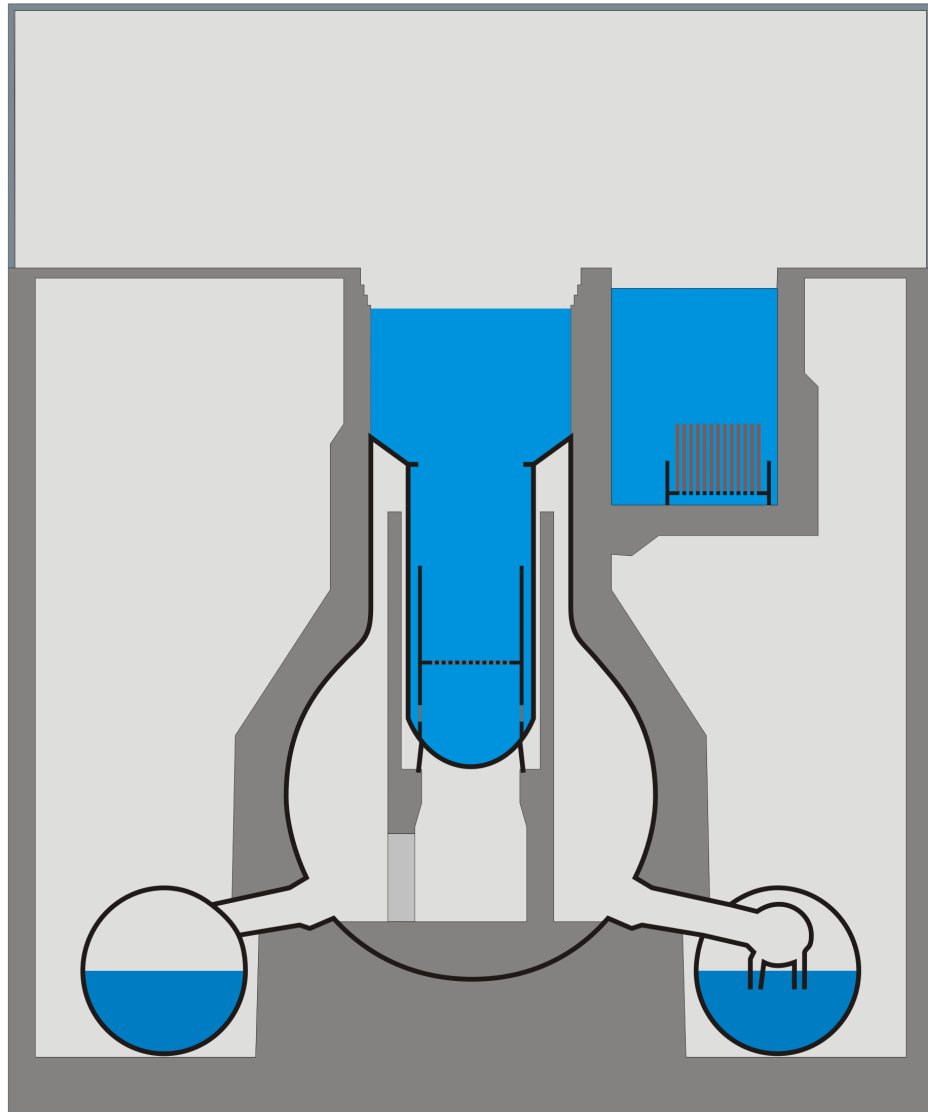
- ▶ ~50km around the plant
 - ◆ Control of Crop / Dairy products
 - ◆ Usage of Iodine pills (Caution, pills can interfere with heart medicine)

The Fukushima Daiichi Incident

4. Spent fuel pools

- ▶ Spent fuel stored in Pool on Reactor service floor
 - ◆ Due to maintenance in Unit 4 entire core stored in Fuel pool
 - ◆ Dry-out of the pools
 - Unit 4: in 10 days
 - Unit 1-3,5,6 in few weeks
 - ◆ **Leakage of the pools due to Earthquake?**

- ▶ Consequences
 - ◆ Core melt „on fresh air “
 - ◆ Nearly no retention of fission products
 - ◆ Large release

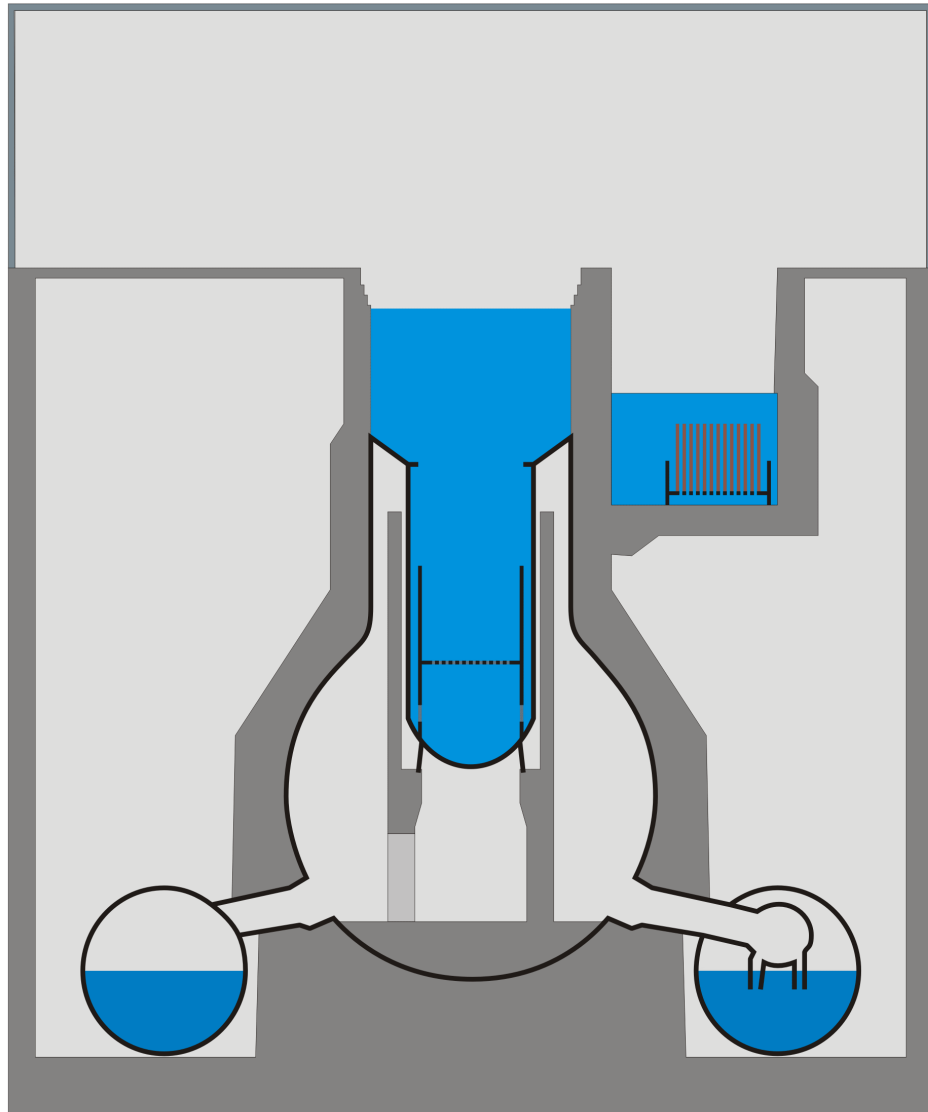


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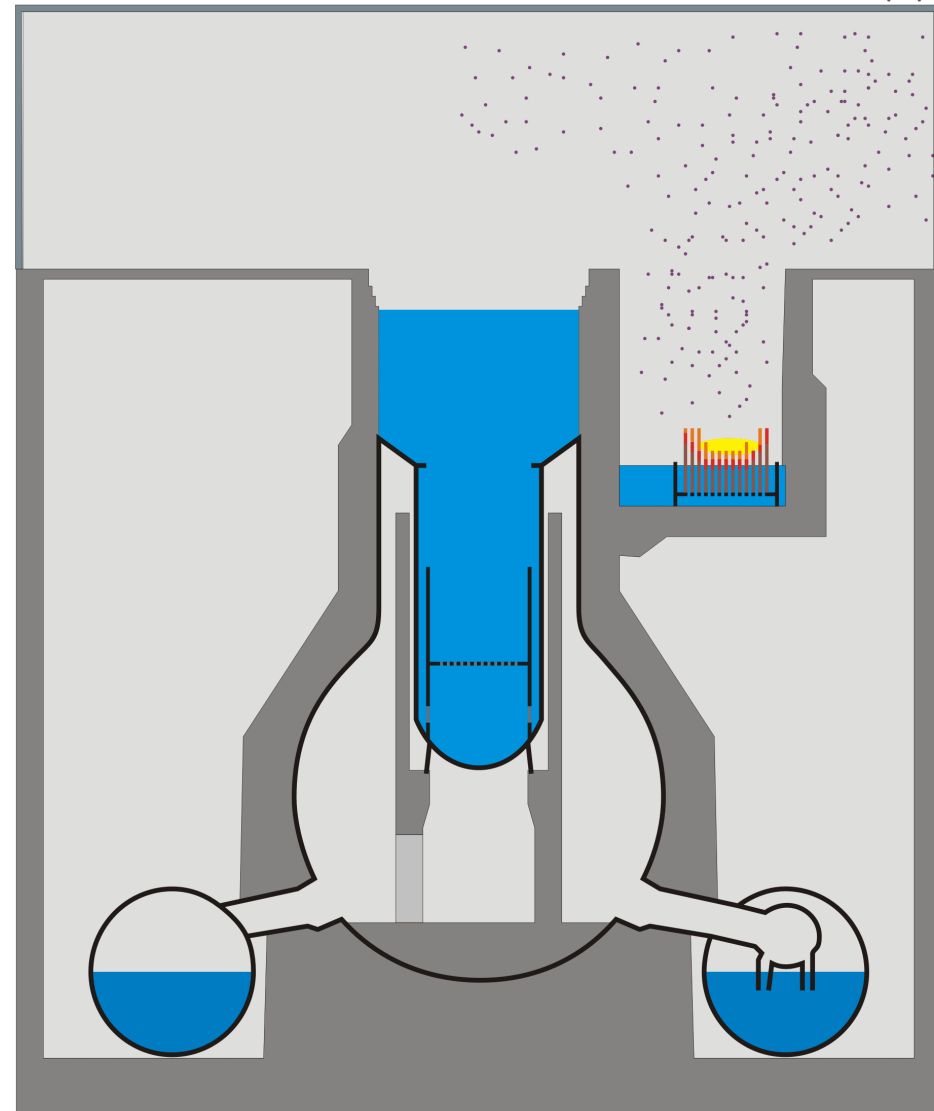
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- ▶ **It is currently unclear if release from fuel pool already happened**



The Fukushima Daiichi Incident

5. Sources of Information



▶ Good sources of Information

- ◆ Gesellschaft für Reaktorsicherheit [GRS.de]
 - Up to date
 - Radiological measurements published
 - German translation of japanese/englisch web pages

- ◆ Japan Atomic Industrial Forum [jaif.or.jp/english/]
 - Current Status of the plants
 - Measurement values of the reactors (pressure liquid level)

- ◆ Tokyo Electric Power Company [Tepco.co.jp]
 - Status of the recovery work
 - Casualties

▶ May too few information are released by TEPCO, the operator of the plant